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Removal of the codeposited carbon layer using He–O glow discharge

C.L. Kunz^{a,*}, R.A. Causey^a, M. Clift^a, W.R. Wampler^b, D.F. Cowgill^a

^a Sandia National Laboratories, P.O. Box 969, Livermore, CA 94550, United States ^b Sandia National Laboratories, Albuquerque, NM, United States

Abstract

In this study we examine the combination of a He–O glow discharge with heating as a possible technique to remove deuterium from TFTR tiles. Samples were cut from a relatively large area containing a uniform codeposited layer of deuterium and carbon. Auger/SEM was used to generate micrographs of each of the samples. The samples were also examined using Rutherford backscattering to determine the near surface composition. Individual samples were then exposed to a He–O glow discharge while being heated. After the exposure, the samples were returned for Auger/SEM and RBS of the same areas examined prior to the exposure. Comparing the samples before and after exposure revealed that the amount of the codeposited layer removed was significantly less than 1 µm. Removal rates this low would suggest that He–O glow discharge with heating is insufficient to remove the thick layers predicted for ITER in a timely fashion. Published by Elsevier B.V.

1. Introduction

The problem of tritium codeposition with carbon to yield excessive tritium inventories presents the ITER fusion reactor with one of its greatest challenges. To mitigate the destructive nature of disruptions and Type 1 ELMs on divertor materials, graphite or carbon composites must be placed in the bottom of the tokamak divertor. With carbon present, nearby surfaces where the rate of deposition is greater than the erosion rate will have codeposited layers of carbon and hydrogen isotopes growing indefinitely. Since ITER will be operating

E-mail address: clkunz@sandia.gov (C.L. Kunz).

with a mixture of tritium and deuterium, the codeposited layer could potentially constrain the operation of ITER by retaining quantities of tritium approaching the safety limit. If we combine the high tritium inventory with the fact that the a-C:H codeposited layer is not stable at elevated temperatures in the presence of air [1], we have a potential environmental hazard in the event of an accidental vacuum loss when the tokamak vessel is hot.

Several techniques have been examined as ways of removing the codeposited layer. These techniques include heating in air or oxygen [1–6], laser heating [7,8], flash lamps [9], and He–O glow discharge [10]. While each of these techniques has shown some success in reducing the quantity of tritium in the codeposited layer, no technique has been identified as the solution to the problem. As an example, heating in

^{*} Corresponding author. Tel.: +1 925 294 6238; fax: +1 925 294 3231.

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air will completely remove the codeposited layer if the temperature is sufficiently high. Unfortunately, ITER will be limited to an upper bake out temperature of approximately 500 K. At this temperature, only about 30% to 50% of the film can be removed during a several hour bake in air or oxygen.

He-O glow discharge at room temperature has been tried in the past as a removal technique for the codeposited layer. Hsu [10] compared the glow discharge removal rate of a codeposited layer using several different types of gases. Of nitrogen, hydrogen, helium, and oxygen, only oxygen (in the form of He-O) was found to have a measurable removal rate. Hsu determined an effective removal rate of approximately 5 atoms of carbon for each oxygen ion striking the layer. While the film was produced by the plasma decomposition of methane, and was therefore a 'soft film' with a significant fraction of weakly bound atoms, this result certainly suggested that He-O might present a reasonable removal process for the codeposited layer. In unpublished experiments [11], Cowgill used a He-O discharge to remove a codeposited layer from a tile taken from the TFTR reactor. These experiments demonstrated a removal rate of $\sim 1.2 \,\mu\text{m/r}$ (about 2.5 carbon atom/O ion) during the initial stage of the experiment, but noted that the removal rate decreased with time. The decrease was attributed to surface texturing. He-O glow discharge was also used directly in the TFTR reactor. For the same experiments, Skinner [12] reported the process to release 50 Ci/h and to be constant with time. This value should be compared to an initial removal rate of 170 Ci/h for deuterium glow discharge, but a steady state release rate of only 10 Ci/h. In a somewhat related series of experiments, Jacob et al. [13] performed a systematic study of the removal of a-C:H layers using electron cyclotron resonance discharges (ECR). Several different species were used for the ECR low-pressure discharges (O₂, D₂, H₂, H_2O , and O_2/H_2), but oxygen always showed the highest removal rates. They noted a codeposited removal rate as high as 1.7 µm/h at 300 K. The authors noted increased yield with increased voltage or temperature, but found the two were not additive.

The essential difference between most the earlier studies and that reported here is that heating during the discharge has been added. An area of net deposition on a graphite tile removed from TFTR prior to the DT campaign was used to provide a relevant codeposited layer. Small samples cut from these tiles were examined prior to and after exposure to a He– O at temperatures from 373 K up to 513 K for 1– 4 h. Changes in the layer thickness and near surface deuterium content were measured.

2. Experimental procedures

Samples used in this experimental program were obtained from a graphite tile removed from the TFTR reactor. The tile had remained in the reactor for several months of deuterium operation, and was covered with a relatively thick codeposited layer. A description of similar tiles removed at the same time and their codeposited layers can be found in [14]. Analysis performed on the tile showed a uniform codeposited layer with a D/C ratio of approximately 0.2. The samples were obtained by slicing approximately a 2 mm thick layer off of the top of the graphite tile. The area of the tile from which the samples were obtained was selected only after microscopy of the entire tile had shown this area to have a relatively uniform codeposited layer.

The samples were first examined microscopically using Auger scanning electron microscopy. They were then analyzed using He³ Nuclear Reaction Analysis (NRA) to determine the deuterium concentration versus depth to a depth of about 1.2 mg/ cm^2 . If the film is assumed to have a density of 1.5 g/cm³, typical of tokamak codeposited layers, the 1.2 mg/cm² would correspond to a thickness of 8 µm's. Additionally, 2.5 MeV proton RBS was used to determine the total thickness of the film. The samples were then exposed to a He-O glow discharge at temperatures varying from 373 K to 513 K (the latter being the highest obtainable temperature in the ITER reactor). The gas consisted of 80% helium and 20% oxygen, and was held at a pressure of 13.2 Pa during the discharge. Slowly pumping on the vacuum vessel at the same time fresh gas was supplied, retained the purity of the gas. A voltage of 470 V was required to generate the discharge, and an incident flux of approximately 10^{19} ions/m² s was obtained. After exposure to the discharge, the samples were then analyzed again using the Auger scanning electron microscopy technique and ion beam analyses.

3. Experimental results and discussion

The amount of codeposited layer removed was determined using two different techniques. The first technique involved direct observation of the sample before and after He–O glow discharge using scanning electron microscopy. The second technique was 2.5 MeV RBS. Additionally, the amount of deuterium removed from the codeposited layer was determined using nuclear reaction profiling. The exposure conditions for the four different samples are shown in Table 1.

The scanning electron microscopy results for sample 4 are shown in Fig. 1; all other samples show less erosion. Even in the more extreme cases of the 443 K exposure for 4 h and the 513 K exposure for 1 h, by comparing the figures the images suggest, qualitatively, that the amount removed was less than 1 μ m. Similar results were seen with the Rutherford backscattering. Fig. 2 shows the RBS results for sample 4; the other three samples show similar behavior.

It is interesting to examine the upper limit of removal as determined by the RBS technique. Sample 4 with the 4 h exposure at 443 K is considered here. For this sample the upper limit of removal is 5% of 13.3 μ m, or 0.667 μ m, this limit was found by fitting the RBS data using SIMNRA [15] with uniform and non-uniform assumptions. Based on a density of 1.5 g/cm³, this distance represents 5× 10¹⁸ carbon atoms removed per cm². During the 4 h exposure, the total fluence was 1.44 × 10¹⁹ ions/ cm². This result would suggest a removal rate of approximately 0.35 carbon atoms/ion. This amount should be compared to the approximately 5 carbon

Table 1Exposure conditions for the four samples

	Temperature (K)	Time (h)	
Sample 1	373	1	
Sample 2	443	1	
Sample 3	513	1	
Sample 4	443	4	



Fig. 2. Rutherford backscattering results for sample 4 after exposure at 443 K for 4 h. The plot shows the fit to the data from SIMNRA using a uniform and non-uniform thickness assumption.

atoms/ion reported by Hsu [10]. There are two possible explanations for this difference in apparent removal rates. One possibility is that the film examined by Hsu was different. That film was produced by the plasma decomposition of methane. Depending upon the deposition conditions, films produced by this technique may have higher hydrogen content and lower density [5]. While it is probable that the two different film types do in fact have different removal rates, another possibility is that the removal rate may start out at a very high rate, but decrease with time. This is exactly what was seen by Cowgill [11] in his unpublished study on He-O removal of codeposited layers. Monitoring of the CO and CO₂ levels in the chamber using a differentially pumped mass spectrometer showed the apparent etch rate of the codeposited layer to substantially decrease with time. Microscopic examination of his samples



Fig. 1. SEM for sample 4 before and after exposure at 443 K for 4 h.

after exposure revealed a 'grass-like' or 'hair-like' structure. Glow discharge produces an ion flux that is normal to the sample surface. It is quite likely that this normal angle of incidence creates the special structure that is resistant to further removal. Part of the justification for the present study using increased temperature was to examine whether the oxygen would react with the individual strands of the structure to result in a continuous, high removal rate. The apparent answer is that this does not occur. The initial versus long-term removal rate may also explain the difference between results seen in the present study and that performed by Jacob et al. [14]. Their maximum removal rate at 300 K was 1.7 µm/h while the present results were limited to only 0.17 μ m/h (0.667 μ m/4 h). Their films were only 300–500 nm thick, and would have been completely removed before the modified structure could be formed.

The above results clearly show that He–O glow discharge is not an effective way to reduce the thickness of a thick codeposited layer. For tritium inventory reduction, He–O glow discharge could still be effective if it were able to selectively remove hydrogen isotopes from the layer. To examine this possibility, nuclear reaction analysis (NRA) of the deuterium in the samples was measured before and after the glow discharge. Due to the similarity of the results, NRA for only one sample is shown (Fig. 3). While the results would appear to suggest



Fig. 3. Nuclear reaction analysis of sample 4 before and after exposure.

some removal of deuterium throughout the first micron or so, it is more likely that the loss of deuterium is restricted to only that very near the surface. Removal of either 5 nm or 500 nm would appear almost identical due to both being less than the depth resolution of the technique. The energetic oxygen ions from the He–O glow should only penetrate about 5 nm into the deposited layer.

4. Conclusion

Experimental results using scanning electron microscopy, Rutherford backscattering, and nuclear reaction analysis of samples exposed to He-O glow discharge during heating all confirm the combination of removal techniques is not an effective way of removing the codeposited carbon/tritium layer produced in fusion reactors. Surface roughening along with near surface removal of hydrogen isotopes does occur, but the etch rate of the layer is too slow for use on the thick films anticipated for ITER and other future fusion devices. Because glow discharge in ITER would require de-energizing the superconducting TF coil, any use of helium and oxygen for codeposited layer removal would come in the form of O-ICR [16]. It is difficult to speculate whether the difference in the angle of incidence for the two techniques would make any difference in the removal rate of the layer.

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